Global calculation of neoclassical impurity transport including the variation of electrostatic potential

Keiji Fujita^[0],[†], S. Satake^{1,2}, R. Kanno^{1,2}, M. Nunami^{1,2}, M. Nakata^{1,2}, J. M. García-Regaña³, J. L. Velasco³ and I. Calvo³

¹The Graduate University for Advanced Studies (SOKENDAI), 322-6 Oroshi-cho, Toki, Japan ²National Institute for Fusion Science, 322-6 Oroshi-cho, Toki, Japan

³Laboratorio Nacional de Fusión, CIEMAT, Avenida Complutense, 40, 28040, Madrid, Spain

(Received 27 March 2020; revised 20 May 2020; accepted 21 May 2020)

Recently, the validity range of the approximations commonly used in neoclassical calculation has been reconsidered. One of the primary motivations behind this trend is observation of an impurity hole in LHD (Large Helical Device), i.e. the formation of an extremely hollow density profile of an impurity ion species, such as carbon C^{6+} , in the plasma core region where a negative radial electric field (E_r) is expected to exist. Recent studies have shown that the variation of electrostatic potential on the flux surface, Φ_1 , has significant impact on neoclassical impurity transport. Nevertheless, the effect of Φ_1 has been studied with radially local codes and the necessity of global calculation has been suggested. Thus, we have extended a global neoclassical code, FORTEC-3D, to simulate impurity transport in an impurity hole plasma including ϕ_1 globally. Independently of the Φ_1 effect, an electron root of the ambipolar condition for the impurity hole plasma has been found by global simulation. Hence, we have considered two different cases, each with a positive (global) and a negative (local) solution of the ambipolar condition, respectively. Our result provides another support that Φ_1 has non-negligible impact on impurity transport. However, for the ion-root case, the radial C⁶⁺ flux is driven further inwardly by Φ_1 . For the electron-root case, on the other hand, the radial particle C^{6+} flux is outwardly enhanced by Φ_1 . These results indicate that how Φ_1 affects the radial particle transport crucially depends on the profile of the ambipolar- E_r , which is found to be susceptible to Φ_1 itself and the global effects.

Key words: fusion plasma, plasma confinement, plasma simulation

1. Introduction

Fusion plasmas contain several ion species other than the fuel ions. Accumulation of such so-called 'impurity ions' in the core region could prevent the achievement of steady-state fusion operation by dilution of the fuel ions or energy loss through radiation. Transport of impurity ions is caused by turbulent and neoclassical processes.

†Email address for correspondence: fujita.keiji@nifs.ac.jp

K. Fujita and others

Although it is thought that the turbulent contribution usually dominates over the neoclassical contribution for bulk plasmas, the contribution of neoclassical transport can also be significant for impurity transport (Burhenn *et al.* 2009). The neoclassical flux of species a is given by

$$\Gamma_a \equiv \left\langle \int d^3 v \boldsymbol{v}_d \cdot \boldsymbol{\nabla} r f_{a1} \right\rangle, \qquad (1.1)$$

where v_d denotes the guiding-centre drift velocity, r the radial coordinate, f_{a1} the nonadiabatic part of the deviation from the local Maxwellian of the species a and $\langle ... \rangle$ the flux surface average, respectively. Γ_a can also be expressed by the linear combination of thermodynamic forces:

$$\Gamma_a = -n_a \sum_b \left[D_{11}^{ab} \left(\frac{n_b'}{n_b} + \frac{Z_b e \Phi'}{T_b} \right) + D_{12}^{ab} \frac{T_b'}{T_b} \right],$$
(1.2)

where n_b denotes the density, T_b the temperature and $Z_b e$ the charge of species *b*, respectively, Φ denotes electrostatic potential and D_{ij}^{ab} the neoclassical transport coefficients and the prime denotes the derivative with respect to the radial coordinate, *r*.

For impurity species, the term

$$-n_z D_{11}^{zz} \frac{Z_z e\Phi'}{T_z} = n_z D_{11}^{zz} \frac{Z_z eE_r}{T_z}$$
(1.3)

tends to dominate with increasing charge number Z_z , where $E_r = -\Phi'$ is the radial electric field and the index z refers to the impurity species. This term drives the impurity inwardly since the coefficient D_{11}^{zz} is positive and the sign of E_r is expected to be negative in typical fusion-relevant plasmas. In axisymmetric systems, however, the terms involving $E_r = -\Phi'$ in (1.2) are cancelled when the sum over the species is taken and the net contribution of E_r to the flux vanishes. The coefficient D_{12}^{zz} is usually positive and it can be larger than the D_{11}^{zb} terms in absolute value. Then, a sufficiently steep temperature gradient can drive the impurity outwardly. This is called the 'temperature screening effect'. On the other hand, in non-axisymmetric systems, the cancellation of E_r terms does not usually occur because neoclassical transport is not intrinsically ambipolar like it is in tokamaks. This is a serious disadvantage of non-axisymmetric devices. Thus, to understand the nature of impurity transport and establish a way to mitigate the impurity accumulation in the core region are crucial tasks for maintaining stable plasma operation.

With this background, a notable phenomenon has been observed in LHD (Large Helical Device). The phenomenon is called 'impurity hole', and it indicates the formation of an extremely hollow density profile of an impurity species in the plasma core region where a negative E_r is expected to exist (Ida *et al.* 2009). The phenomenon had been first observed for carbon C⁶⁺ and in a later study (Yoshinuma, Ida & Yokoyama 2010) it was reported that the same hollow structure can also be formed in the density profile of neon Ne¹⁰⁺. The study has also shown that the density in the core becomes hollower for heavier impurities. These observations contradict the prediction of the standard neoclassical transport theory that medium or heavy impurities are expected to be driven inwardly by the negative E_r . Gyro-kinetic approaches have also been taken, but they indicate that the turbulent C⁶⁺ flux also directs inwardly in ion-root plasmas with hollow density profiles. It is also shown

that, in such plasmas, the total neoclassical flux can be too large to satisfy the particle balances between turbulent and neoclassical contributions (Nunami et al. 2016, 2017, 2019, 2020). Thus, the validity range of the approximations commonly used for neoclassical simulations has been reconsidered. For example, Helander et al. (2017) have pointed out that describing collision processes only with the pitch angle scattering operator, as conventionally done, is inadequate for studying impurity transport and, by retaining the friction force part, they have shown the cancellation of the E_r terms and hence temperature screening is in fact possible even in non-axisymmetric systems under certain conditions (see also Newton et al. 2017). In order for the cancellation to take place in a helical plasma, the plasma needs to be in a relevant mixed collisionality, where a highly collisional impurity species exists in the low collisional background ion. In addition, the variation of electrostatic potential over the flux surface, $\Phi_1 \equiv \Phi - \Phi_0$, is required to be sufficiently small (Buller et al. 2018; Calvo et al. 2018a); here $\Phi_0 = \Phi_0(r)$ is the flux function part of the electrostatic potential. In Velasco et al. (2019), an experimental case in which the temperature screening may be occurring is discussed with numerical analysis.

Independently of those works, Φ_1 itself has been thought to have a non-negligible impact on impurity transport. Although Φ_1 itself is usually smaller by one or two orders of magnitude than the uniform part (Calvo *et al.* 2018*b*), the impact of Φ_1 in the guiding-centre motions relative to the impact of the magnetic field through the magnetic drift or the mirror force becomes larger proportionally to the charge of the species. Thus, while the effect of Φ_1 is usually small for the main ion or electron, it can be essential for highly charged impurities. While measuring such a small variation in electrostatic potential is still challenging in LHD, it is already technically feasible in the smaller device TJ-II and the potential variation has actually been detected in such experiments (Pedrosa et al. 2015; García-Regaña et al. 2018; Estrada et al. 2019). Also, by several simulation studies, it has already been shown that Φ_1 can have substantial impact on neoclassical impurity transport (see e.g. García-Regaña et al. 2013, 2017; Calvo et al. 2018a; Mollén et al. 2018; Velasco et al. 2018). However, all of those calculations have been performed with radially local codes, i.e. the models in which the radial drift in the guiding-centre motion is neglected and dynamics on each flux is separated.

In most of the local models, the magnetic drift, v_m , is entirely dropped from the equation for the guiding-centre orbit in the configuration space. However, the validity of this approximation for low-collisionality cases is ensured only when the $E \times B$ drift, v_{E} , is large enough compared with the magnetic drift. When E_r is close to zero, the helically trapped particles lose their mobility on the flux surface and the contribution of the trapped particles to radial transport significantly increases. Consequently, radial fluxes in the conventional local models show a strong peak around $E_r = 0$. Such an excessive transport can be largely moderated by the inclusion of the tangential component of the magnetic drift. The tangential v_m moves the trapped particles along the flux surface and can detrap them without collision. Also, the peaking point is shifted from where $E_r = 0$ to where the particle average of $v_d \cdot \nabla \alpha \equiv (v_m + v_E) \cdot \nabla \alpha$ vanishes on the flux surface, where the overbar denotes the orbit average and α labels magnetic field lines. As the result, the difference in the radial particle or energy flux of hydrogen ions between calculations with and without the tangential v_m can be an order of magnitude (Matsuoka et al. 2015; Huang et al. 2017; Velasco et al. 2020). By evaluating Φ_1 and its impact on impurity transport using a local model with the tangential v_m , Velasco *et al.* (2018) also found that the phase structure of Φ_1 can be substantially modified by the tangential v_m . Further, in unoptimised devices such

as LHD under the conditions being discussed, the radial component of v_m can be non-negligible as well (Calvo *et al.* 2017). In LHD cases, it has been shown that the hydrogen fluxes obtained by calculation with the full v_m can be a few times smaller than those obtained by calculation only with the tangential v_m (Matsuoka *et al.* 2015; Huang *et al.* 2017). The effect of the radial v_m appears in the structure of Φ_1 as well (Fujita *et al.* 2019). Thus, in order to investigate the structure of Φ_1 and its impact on transport in low-collisionality LHD plasmas, global effects should not be neglected.

Recently, we have made two major extensions of a global neoclassical simulation code, FORTEC-3D. The first is to enable it to evaluate Φ_1 (Fujita *et al.* 2019) and include its effect in a drift-kinetic equation. The second is the implementation of a cross ion-species collision operator to perform multi-ion-species calculation. The collision operator retains both test-particle and field-particle parts and the effects of the momentum and energy exchanges by friction are included (Sugama, Watanabe & Nunami 2009; Satake *et al.* 2020). Thus, this newly extended code can be used to investigate the effect of Φ_1 on impurity transport including the global effects. Verification of the numerical schemes in the multi-species version of FORTEC-3D will appear in a separate paper. In this paper, this extended version of the code is applied to impurity transport in LHD plasmas in which hollow carbon density profiles are formed. This paper is organised as follows: the theoretical model used for this study is introduced in § 2. The setup for the calculation is presented in § 3. The result is shown and discussed in § 4. Finally, a conclusion is drawn in § 5.

2. Theoretical model and numerical method

In this section, the theoretical model used in FORTEC-3D is presented. FORTEC-3D is a Monte Carlo δf code which solves a drift-kinetic equation with full linearised Landau collision operator without employing the radially local approximation (Satake, Kanno & Sugama 2008; Satake *et al.* 2010).

2.1. Drift-kinetic equation

When the guiding-centre coordinates X, the parallel velocity v_{\parallel} and the magnetic moment $\mu = m_a v_{\perp}^2/(2B)$ are chosen as the phase-space variables, where $B = |\mathbf{B}|$ and v_{\perp} is the perpendicular velocity, the guiding-centre equations of motion are given by

$$\dot{\boldsymbol{X}} = \boldsymbol{v}_{\parallel}\boldsymbol{b} + \frac{1}{Z_a e B_{\parallel}^*} \boldsymbol{b} \times (m_a \boldsymbol{v}_{\parallel}^2 \boldsymbol{b} \cdot \boldsymbol{\nabla} \boldsymbol{b} + \mu \boldsymbol{\nabla} \boldsymbol{B} + Z_a e \boldsymbol{\nabla} \boldsymbol{\Phi}), \qquad (2.1)$$

$$\dot{v}_{\parallel} = -\frac{1}{m_a v_{\parallel}} \dot{X} \cdot (\mu \nabla B + Z_a e \nabla \Phi), \qquad (2.2)$$

$$\dot{\mu} = 0, \tag{2.3}$$

where $\mathbf{b} = \mathbf{B}/B$, $\mathbf{B}^* = \nabla \times \mathbf{A}^*$ is the corrected magnetic field with the guiding-centre vector potential $\mathbf{A}^* = \mathbf{A} + m_a v_{\parallel} \mathbf{b}/(Z_a e)$, $B^*_{\parallel} = \mathbf{B}^* \cdot \mathbf{b}$ and the overdot denotes the total time derivative $\dot{A} \equiv dA/dt$ of any function A. It is assumed that $\partial A/\partial t = \partial A^*/\partial t = 0$ in deriving the equations above.

Splitting the distribution function of species *a* as $f_a = f_{a0} + f_{a1}$, where $f_{a1} \ll f_{a0}$, FORTEC-3D solves the equation for f_{a1} . The lowest-order distribution function f_{a0} is given by

$$f_{a0} = f_{aM} e^{-Z_a e \Phi_1 / T_a}, \tag{2.4}$$

with the local Maxwellian

$$f_{aM} = n_{a0} \left(\frac{m_a}{2\pi T_a}\right)^{3/2} \exp\left[-\frac{m_a v_{\parallel}^2}{2T_a} - \frac{\mu B}{T_a}\right].$$
 (2.5)

The first-order drift-kinetic equation for f_{a1} is then given by

$$\frac{\partial f_{a1}}{\partial t} + \dot{X} \cdot \nabla f_{a1} + \dot{v}_{\parallel} \frac{\partial f_{a1}}{\partial v_{\parallel}} - C_{TP}(f_{a1}) = C_{FP}(f_{a0}) + \frac{Z_a e}{T_a} \frac{\partial \Phi_1}{\partial t}$$
$$- \dot{X} \cdot \nabla r \left[\frac{n'_{a0}}{n_{a0}} + \frac{Z_a e \Phi'_0}{T_a} + \left(\frac{m_a v_{\parallel}^2}{2T_a} + \frac{\mu B}{T_a} - \frac{3}{2} + \frac{Z_a e \Phi_1}{T_a} \right) \frac{T'_a}{T_a} \right] f_{a0}, \quad (2.6)$$

where n_{a0} and T_a are flux functions, m_a denotes the mass of the species a and $C_{TP}(f_{a1})$ and $C_{FP}(f_{a0})$ are the test-particle part and the field-particle part of the linearised collision operator, respectively,

$$C_{TP}(f_{a1}) = \sum_{b} C_{TP}^{ab}(f_{a1}, f_{b0}), \qquad (2.7)$$

$$C_{FP}(f_{a0}) = \sum_{b} C_{FP}^{ab}(f_{a0}, f_{b1}).$$
(2.8)

Strictly speaking, however, f_{a0} is approximated with f_{aM} in the linearised collision operator for the present work. Also, the recent attempts to extend the collision operator to include higher-order terms which can be important for impurity transport (Sugama *et al.* 2019; Calvo *et al.* 2020) are not considered. The equations (2.1)–(2.6) are reduced to the equations for a case without Φ_1 by setting $\Phi_1 = 0$. A derivation of the drift-kinetic equation (2.6) is provided in appendix A.

Note that in this model, the term corresponding to the so-called $E \times B$ drift

$$\boldsymbol{v}_E \equiv \frac{1}{B_{\parallel}^*} \boldsymbol{b} \times \nabla \boldsymbol{\Phi} \tag{2.9}$$

is species-dependent due to the denominator

$$B_{\parallel}^{*} = B + \frac{m_{a}v_{\parallel}}{Z_{a}e}\boldsymbol{b} \cdot \boldsymbol{\nabla} \times \boldsymbol{b}$$
(2.10)

and FORTEC-3D does solve (2.1)–(2.3) as written here so that the phase-space volume is conserved (Littlejohn 1983). However, the second term in (2.10) is negligible as far as the low-beta approximation

$$\nabla \times \boldsymbol{b} \simeq \frac{\boldsymbol{B} \times \nabla \boldsymbol{B}}{B^2} \tag{2.11}$$

holds. Then, v_E can be approximated as

$$\boldsymbol{v}_E \simeq \frac{1}{B} \boldsymbol{b} \times \boldsymbol{\nabla} \boldsymbol{\Phi} \tag{2.12}$$

and the value of the $E \times B$ drift becomes species-independent. Also, it does not affect the basic property that the lowest-order distribution function does not generate radial particle flux:

$$\Gamma_{a0} \equiv \left\langle \int d^3 v (\boldsymbol{v}_m + \boldsymbol{v}_E) \cdot \boldsymbol{\nabla} r f_{a0} \right\rangle = 0.$$
(2.13)

The verification of (2.13) is provided in appendix A.

K. Fujita and others

2.2. Expressions in the Boozer coordinate system

The Boozer coordinate system (Boozer 1981) is used in FORTEC-3D. In this coordinate system, the covariant representation of the magnetic field is given by

$$\boldsymbol{B} = \nabla \boldsymbol{\psi} \times \nabla \boldsymbol{\theta} + \iota(r) \nabla \boldsymbol{\zeta} \times \nabla \boldsymbol{\psi} = I(r) \nabla \boldsymbol{\theta} + G(r) \nabla \boldsymbol{\zeta} + \boldsymbol{\beta}^* \nabla \boldsymbol{\psi}, \qquad (2.14)$$

where ι is the rotational transform, ψ is the toroidal magnetic flux divided by 2π , I is related to the toroidal current inside the flux surface at r and G is related to the poloidal current outside the surface, respectively. Since β^* is related to the Pfirsch–Schlüter current (Boozer 1981) and can be estimated as $O(\beta/a)$, where β and a are the plasma beta and the minor radius, respectively, we assume low- β plasmas and therefore the term $\beta^* \nabla \psi$ in (2.14) is ignored in this work. This approximation allows us to put the guiding-centre equations of motion in canonical form. Although we will not write down the equations in canonical form explicitly here, this property has an advantage of ensuring the properties of Hamiltonian systems such as Liouville's theorem. See § 3.2 of White (2014) and references therein for a detailed discussion of the construction of the Hamiltonian formulation of the guiding-centre motion. The contravariant representation is given by

$$\boldsymbol{B} = \frac{1}{\sqrt{g}} \frac{\mathrm{d}\psi}{\mathrm{d}r} (\iota \boldsymbol{e}_{\theta} + \boldsymbol{e}_{\zeta}), \qquad (2.15)$$

where e_i is the unit vector in the *j*-direction and the Jacobian \sqrt{g} is given by

$$\sqrt{g} = \frac{\mathrm{d}\psi}{\mathrm{d}r} \frac{G + \iota I}{B^2} = \frac{(\psi' G + \chi' I)}{B^2},$$
 (2.16)

with χ the poloidal magnetic flux divided by 2π .

In this coordinate system, the guiding-centre equations of motion are expressed as follows (a derivation is provided in A.2):

$$\frac{\mathrm{d}r}{\mathrm{d}t} = \frac{\delta}{\gamma} \left(I \frac{\partial B}{\partial \zeta} - G \frac{\partial B}{\partial \theta} \right) + \frac{Z_a e}{\gamma} \left(I \frac{\partial \Phi}{\partial \zeta} - G \frac{\partial \Phi}{\partial \theta} \right), \qquad (2.17)$$

$$\frac{\mathrm{d}\theta}{\mathrm{d}t} = \frac{G}{\gamma} (\delta B' + Z_a e \Phi') + \frac{v_{\parallel} B}{\gamma} \left(Z_a e \chi' - \frac{m_a v_{\parallel}}{B} G' \right), \qquad (2.18)$$

$$\frac{\mathrm{d}\zeta}{\mathrm{d}t} = -\frac{I}{\gamma} (\delta B' + Z_a e \Phi') + \frac{v_{\parallel} B}{\gamma} \left(Z_a e \psi' + \frac{m_a v_{\parallel}}{B} I' \right), \qquad (2.19)$$

$$\frac{\mathrm{d}v_{\parallel}}{\mathrm{d}t} = -\frac{v_{\parallel}}{B\gamma} \left[Z_{a}e \left(I \frac{\partial B}{\partial \zeta} - G \frac{\partial B}{\partial \theta} \right) \Phi' - \mu \left(I \frac{\partial \Phi}{\partial \zeta} - G \frac{\partial \Phi}{\partial \theta} \right) B' \right]
+ \frac{Z_{a}eB}{m_{a}\gamma} \left[\left(\frac{m_{a}v_{\parallel}}{Z_{a}eB}G' - \chi' \right) \left(\mu \frac{\partial B}{\partial \theta} + Z_{a}e \frac{\partial \Phi}{\partial \theta} \right)
- \left(\frac{m_{a}v_{\parallel}}{Z_{a}eB}I' + \psi' \right) \left(\mu \frac{\partial B}{\partial \zeta} + Z_{a}e \frac{\partial \Phi}{\partial \zeta} \right) \right],$$
(2.20)

where

$$\delta \equiv \mu + \frac{m_a v_{\parallel}^2}{B} \tag{2.21}$$

and

$$\gamma \equiv Z_a e B B^*_{\parallel} \sqrt{g} \tag{2.22}$$

are defined. δ here is not to be confused with the delta function or some ordering parameter. The flux surface average of an arbitrary function A is expressed as

$$\langle A \rangle = \frac{1}{V'} \oint \sqrt{g} \, \mathrm{d}\theta \, \mathrm{d}\zeta A, \qquad (2.23)$$

where

$$V' = \oint \sqrt{g} \,\mathrm{d}\theta \,\mathrm{d}\zeta \,. \tag{2.24}$$

The radial particle flux is thus calculated by

$$\Gamma_a = \frac{1}{V'} \oint \sqrt{g} \,\mathrm{d}\theta \,\mathrm{d}\zeta \,\int \,\mathrm{d}^3 v i f_{a1}. \tag{2.25}$$

2.3. Two-weight δf method

The two-weight δf scheme (Brunner, Valeo & Krommes 1999; Wang *et al.* 1999) is employed in FORTEC-3D. The microscopic dynamics is simulated by marker particles which evolve according to the equations of motion (2.1)–(2.3). In the two-weight δf method, the distribution function is split into two parts as $f = f_0 + f_1$, where f_0 is an analytically known part and f_1 is the deviation from that. In this study, the adiabatic response to Φ_1 is included in the f_0 part (2.4). See García-Regaña *et al.* (2017) for a discussion of different choices of f_0 . The species index is omitted for simplicity in this subsection.

The distribution functions are represented by the products of the marker particle distribution $g(\mathbf{Z}, t)$ and the weight fields $W(\mathbf{Z}, t)$ and $P(\mathbf{Z}, t)$, respectively, as

$$f_0(\mathbf{Z}, t) = P(\mathbf{Z}, t)g(\mathbf{Z}, t), \qquad (2.26)$$

$$f_1(\mathbf{Z}, t) = W(\mathbf{Z}, t)g(\mathbf{Z}, t).$$
(2.27)

Each marker particle is assigned two weights w_i and p_i , which are defined as the values of the weight fields W and P at the phase position Z_i , respectively:

$$w_i(t) = W(\mathbf{Z}_i(t), t), \qquad (2.28)$$

$$p_i(t) = P(\mathbf{Z}_i(t), t), \qquad (2.29)$$

where i is the particle label. The weights also evolve in time following the equations

$$\dot{p}_i = \frac{p_i}{f_0} \dot{\mathbf{Z}} \cdot \frac{\partial}{\partial \mathbf{Z}} f_0 - \frac{Ze}{T} \frac{\partial \Phi_1}{\partial t} p_i, \qquad (2.30)$$

$$\dot{w}_i = -\frac{p_i}{f_0} \left(\dot{\mathbf{Z}} \cdot \frac{\partial}{\partial \mathbf{Z}} - C_{\rm FP} \right) f_0 + \frac{Ze}{T} \frac{\partial \Phi_1}{\partial t} p_i, \qquad (2.31)$$

which are evaluated at $\mathbf{Z} = \mathbf{Z}_i(t)$. We thus can solve the drift-kinetic equation for f_1 by integrating the evolution equations (2.1)–(2.3), (2.30) and (2.31).

K. Fujita and others

2.4. Evaluation of electrostatic potential

The electrostatic potential can be decomposed as $\Phi = \Phi_0 + \Phi_1$. The first part is a flux function $\Phi_0 = \Phi_0(r)$ and generates a radial electric field $E_r = -\Phi'_0$. The radial electric field E_r is determined so that it satisfies the ambipolar condition

$$\sum_{a} Z_a \Gamma_a = 0. \tag{2.32}$$

In FORTEC-3D, $E_r = -\Phi'_0$ can be given as a fixed input parameter or it also can be internally solved by the equation (2*a*) in Satake *et al.* (2008).

The second part $\Phi_1 = \Phi_1(r, \theta, \zeta)$ is the non-flux function part. It is assumed that $\langle \Phi_1 \rangle = 0$ in the present study. See appendix B for the justification of this assumption. The non-uniform part Φ_1 is determined by the following argument. From (2.4), the density of species *a* takes the form

$$n_a = n_{a0} \exp\left(-Z_a e \Phi_1 / T_a\right) + n_{a1} \tag{2.33}$$

and imposing quasi-neutrality up to the first order yields the expression

$$\Phi_{1} = \frac{1}{e} \left(\sum_{I} Z_{I}^{2} \frac{n_{I0}}{T_{I}} + \frac{n_{e0}}{T_{e}} \right)^{-1} \sum_{I} Z_{I} n_{I1} \\
= \frac{1}{e} \left(\sum_{I} Z_{I}^{2} \frac{n_{I0}}{T_{I}} + \frac{n_{e0}}{T_{e}} \right)^{-1} \sum_{I} Z_{I} \int d^{3} v f_{I1},$$
(2.34)

where the subscript *I* refers to the ion species. In deriving the above expression, $Z_a e \Phi_1/T_a \ll 1$ is assumed for all species. As we shall see below, this assumption is reasonable for the present study. However, the assumption does not hold for high-*Z* impurities. Also, the adiabatic response of electrons is assumed, i.e. $n_{e1} = 0$. The spatial structure of Φ_1 is thus determined by the superposition of the ion density variations n_{I1} , weighted by the charge number Z_I . Note here that in some studies, Φ_1 is evaluated by considering only the contribution of the main ion. When the contribution of impurities is taken into account as (2.34), the value of Φ_1 usually becomes smaller since the impurity contributions are included in the factor $\sum_I Z_I^2 n_{I0}/T_I$ in the denominator. In this study, Φ_1 is evaluated during the self-consistent calculation of the set of equations (2.1)–(2.3), (2.6) and (2.34). It has been pointed out that the contribution of kinetic electrons to Φ_1 can be significant, especially in electron-root plasmas (García-Regaña *et al.* 2018). However, it is numerically too costly for the present version of FORTEC-3D to solve kinetic electrons with ion species together.

2.5. Evaluation of the flux driven by Φ_1

In the Boozer coordinate system, the radial particle flux driven by Φ_1 through the $E \times B$ drift, v_{E1} , is expressed as

$$\Gamma_{v_{E1}}(r) = \frac{1}{V'(r)} \oint \sqrt{g}(X) \, \mathrm{d}\theta \, \mathrm{d}\zeta \int \mathrm{d}^3 v \frac{1}{B(X)B^*_{\parallel}(X, \boldsymbol{v})\sqrt{g}(X)} \\ \times \left(I(r)\frac{\partial \Phi(X)}{\partial \zeta} - G(r)\frac{\partial \Phi(X)}{\partial \theta}\right) f_{a1}(X, \boldsymbol{v}), \tag{2.35}$$

where $X = (r, \theta, \zeta)$ and $v = (v_{\parallel}, \mu)$. This quantity can be evaluated in two different ways. One is to simply perform the integral and the other is to use Fourier series expansion. If the low-beta approximation

$$\frac{1}{B(X)B^*_{\parallel}(X, v)} \simeq \frac{1}{B^2(X)}$$
(2.36)

is used, the velocity integral can readily be performed and we obtain

$$\Gamma_{v_{E1}}(r) = \frac{1}{V'(r)} \oint d\theta \, d\zeta \, \bar{v}_{E1}(\boldsymbol{X}) n_{a1}(\boldsymbol{X}), \qquad (2.37)$$

where we have defined

$$\bar{v}_{E1} \equiv \frac{1}{B^2} \left(I \frac{\partial \Phi}{\partial \zeta} - G \frac{\partial \Phi}{\partial \theta} \right) \simeq \sqrt{g} \boldsymbol{v}_{E1} \cdot \boldsymbol{\nabla} r.$$
(2.38)

Thus, if we expand \bar{v}_{E1} and n_{a1} in Fourier series, respectively, and put them into (2.37), only the products of the same modes survive and the equation is formally reduced to the following form:

$$\Gamma_{v_{E1}}(r) = \frac{1}{V'(r)} \sum_{m,n} C_{m,n} \left[\tilde{v}_{E1}^{m,n(c)}(r) \tilde{n}_{a1}^{m,n(c)}(r) + \tilde{v}_{E1}^{m,n(s)}(r) \tilde{n}_{a1}^{m,n(s)}(r) \right],$$
(2.39)

where $C_{m,n}$ are normalisation coefficients and $\tilde{v}_{E1}^{m,n(c)}$, $\tilde{v}_{E1}^{m,n(s)}$, $\tilde{n}_{a1}^{m,n(c)}$ and $\tilde{n}_{a1}^{m,n(s)}$ are the Fourier coefficients of cosine- and sine-(m, n) modes of \bar{v}_{E1} and n_{a1} , respectively. In the present article, Fourier coefficients of a quantity A(X) are defined as follows:

$$\tilde{A}^{m,n(c)}(r) = \frac{1}{2\pi^2} \iint d\theta \, d\zeta A(\mathbf{X}) \cos(m\theta - nN\zeta),
\tilde{A}^{m,n(s)}(r) = \frac{1}{2\pi^2} \iint d\theta \, d\zeta A(\mathbf{X}) \sin(m\theta - nN\zeta),$$
(2.40)

where N is the toroidal period number (N = 10 for LHD). Note that the flux driven by the magnetic drift, v_m , cannot be put in the form of (2.37) and therefore also not in the form of (2.39) since v_m essentially depends on the velocity variables (and the dependence cannot be approximated away).

Although the first method is more simple and straightforward for evaluating $\Gamma_{v_{E1}}$, this second method has an advantage that it enables us to see how each mode contributes to the additional flux. The C⁶⁺ flux will be evaluated by these two different methods below.

3. Setup for the ion transport calculation

We consider an LHD hydrogen plasma which contains two different impurity species, helium He²⁺ and carbon C⁶⁺. Electrons are assumed to be adiabatic in solving the transport of ion species and in the quasineutrality condition. The standard LHD configuration with a major radius $R_0 = 3.7$ m and a minor radius a = 0.62 m is considered. The magnetic field strength and its Fourier spectrum are represented in figure 1. The density and temperature profiles are shown in figures 2 and 3. One of the features which characterises an impurity hole plasma is the steepness of its



FIGURE 1. The magnetic field strength on the flux surface at r/a=0.2 (a), 0.5 (b) and 0.8 (c) and the major Fourier components of the magnetic field (d). The value of the cosine-(0, 0) component in (d) is adjusted by subtracting 2.5 for visualisation purpose. θ and ζ are poloidal and toroidal angle coordinates in the Boozer coordinate system, respectively, and N = 10.

ion temperature gradient. This profile has been used for several impurity hole studies (Nunami *et al.* 2016, 2017, 2019, 2020; Mollén *et al.* 2018). In this profile, impurity ions comprise a relatively large portion of the total ion density. The effective charge $Z_{\text{eff}} = \sum_{I} n_{I} Z_{I}^{2}/n_{e}$ varies from 1.85 to 2.14 along the minor radius and its average value over the minor radius is 2.02. Although the trace approximation of the impurities is thus not appropriate, impurity contributions are treated as trace quantities in the evaluation of the ambipolar radial electric field in this study for the reason of computational cost. Since the ambipolar radial electric field is determined by the condition (2.32), the approximation can still be justified if the impurity contribution to the total ion particle flux turns out to be small enough compared with the main ion contribution, the ambipolar- E_r is to be modified from the ones obtained by the pure H calculations. Although the detail of how the presence of impurities affects the ambipolar condition depends on the case, positive impurity fluxes usually enhance the ambipolar- E_r in the negative direction.

In figure 4, the normalised collisionality, ν_a^* , of each ion species is plotted. Here the quantity is defined by

$$\nu_a^* \equiv \sum_b \frac{R\nu_{ab}/\epsilon^{3/2}}{\iota v_{Ta}},\tag{3.1}$$



FIGURE 2. Radial profiles of densities.



FIGURE 3. Radial profiles of temperatures. All the ion species are assumed to be in thermal equilibrium with each other.



FIGURE 4. The normalised collisionality, ν_a^* , of H¹⁺ (red), He²⁺ (green), C⁶⁺ (blue) and the lower bound of the Pfirsch–Schlülter regime (magenta).

with the collision frequency between species a and b

$$v_{ab} = \frac{Z_a^2 Z_b^2 e^4 n_b \ln \Lambda}{3\pi^{3/2} \varepsilon_0^2 m_a^2 v_{T_a}^3},$$
(3.2)

where $\epsilon = r/R$ is the inverse aspect ratio, $v_{Ta} = \sqrt{2T_a/m_a}$ is the thermal velocity of species *a*, ln *A* is the Coulomb logarithm and ε_0 is the vacuum permittivity, respectively. The sum in (3.1) is taken over all the ion species including the species *a* itself. In the figure, the magenta line represents the lower bound of the



FIGURE 5. Radial profiles of the ambipolar radial electric fields for the ion-root case obtained by a DKES/PENTA local calculation (solid blue line) and the electron-root case obtained by a FORTEC-3D global calculation (red line), respectively. An electron-root found by the local calculation is also plotted (dashed blue line).

Pfirsch–Schlülter regime. The banana regime in the axisymmetric limit corresponds to

$$\nu_a^* < 1 \tag{3.3}$$

and the intermediate region between $\nu_a^* = 1$ and the magenta line to the plateau regime. In the present case, H^{1+} and He^{2+} are in the $1/\nu$ or $\sqrt{\nu} \sim \nu$ regime. C^{6+} is not so highly collisional to be in the Pfirsch–Schlülter regime and belongs in the plateau regime.

3.1. Ambipolar radial electric field

The ambipolar condition has been determined by two different (sets of) numerical codes. One is a set of local codes DKES/PENTA and the other is the global code FORTEC-3D. Both FORTEC-3D and DKES/PENTA determined the ambipolar condition by assuming a pure H plasma. Both solutions are represented in figure 5. The red line corresponds to the global solution and the solid blue line to the local solution. As can be seen, those solutions are completely different. The sign of the global solution is positive. In other words, the global calculation indicates that the plasma is in electron-root, not in ion-root. It is not necessarily surprising that the global calculation yields different solutions from local ones. As mentioned in the introduction, it is known that the magnetic drift (even only the tangential component of it) can significantly change the dependence of neoclassical transport on E_r (Matsuoka et al. 2015; Huang et al. 2017; Velasco et al. 2020). In Velasco et al. (2017), the ambipolar conditions for such impurity hole plasmas are investigated based on the collisionality and it is argued that, even if an impurity hole plasma is in ion-root, the plasma profiles are very close to the condition for E_r to become positive. In fact, as represented by the dashed line in figure 5, a positive solution is found by the local codes as well. The solution is stable and physically possible. Its existence suggests that the gap between the local and global solutions is not as large as it appears. Yet, we adopt the ion-root (represented by the solid blue line) as our 'local solution' since similar solutions are used in previous local studies. We thus investigate the difference of Φ_1 structure and its impact on impurity transport between the cases, each with the local and the global solutions, respectively.

However, whether the sign of the ambipolar- E_r is positive or negative is the most crucial issue in the impurity hole study. If the sign of ambipolar- E_r turns out to be positive in real impurity hole plasmas, the scenario is completely changed and the impurity hole phenomenon could be explained by standard neoclassical theory arguments. Nunami *et al.* (2019, 2020) have intensively investigated the condition for both solutions by considering the contributions of turbulence and external torque. The result of the study also seems to support the possibility of the positive solution.

There is measurement data on the ambipolar- E_r profile in an impurity hole plasma and it indicates that the sign of the ambipolar- E_r near the magnetic axis is indeed negative (Ido *et al.* 2010). However, it also shows that the sign of E_r transits to positive at some outer radius. Impurity hole has been observed in plasmas with several different n-T profiles (Nakamura *et al.* 2017), but the measurement data on ambipolar- E_r has been reported only for the case mentioned above. The n-T profile used in this study is close to but not the same as that in the measured case. Thus, our global solution for the ambipolar condition does not necessarily contradict the existing experimental data.

Since the following concerns remain, however, we cannot simply accept the electron-root as the correct solution yet. First, the positive solution is, as mentioned above, obtained by a single ion-species calculation and the contribution of impurities to the ambipolar condition is not considered. Second, the Φ_1 effect is not included in the ambipolar condition as well. In contrast, negative solutions similar to our local solution have been obtained by self-consistent calculations for multi-ion-species plasmas including Φ_1 (Mollén *et al.* 2018). How Φ_1 affects the radial particle fluxes in our electron-root case will be examined below.

In any case, the primary purpose of this study is not to examine the ambipolar condition, but to apply the newly extended code to impurity transport in a real helical configuration. Thus, we hold both possibilities here and investigate two cases corresponding to each solution, respectively. Both cases have the same background parameters (such as the magnetic field configuration, densities and temperatures) except for the radial electric field.

4. Simulation results

In this section, calculation results of the cases with and without Φ_1 for each ion-root case and electron-root case are shown and compared to see the impact of Φ_1 on the radial particle transport. The result of the ion-root case, which is mainly focused on, is shown first and the result of the electron-root case follows.

4.1. Ion-root case

In figure 6, the radial profiles of particle fluxes are compared between the cases with and without Φ_1 . The red lines represent the fluxes for the case without Φ_1 and the green lines for the case with Φ_1 . Before discussing the impact of Φ_1 , it is worth commenting on the result without Φ_1 . The H¹⁺ flux obtained by our global calculation is smaller by one order than that obtained by local calculations for similar plasmas (Mollén *et al.* 2018). On the other hand, the values of the He²⁺ and C⁶⁺ fluxes in our global result are of the same order as those in the local study. In this ion-root case, the plasma profile belongs in a parameter region where the discrepancy in the H¹⁺ fluxes between global and local (neglecting the entire v_m) calculations becomes the largest (Matsuoka *et al.* 2015; Huang *et al.* 2017). This significant difference in the main ion flux directly affects the ambipolar condition. Thus, it is natural that the



FIGURE 6. Radial particle fluxes of H¹⁺ (a), He²⁺ (b) and C⁶⁺ (c) for the ion-root case without (red) and with (green) Φ_1 , respectively.



FIGURE 7. From left, structures of Φ_1 , $n_{H,1}$, $n_{He,1}$ and $n_{C,1}$ on the flux surface at r/a=0.2 for the ion-root case. Figures (a-d) represent the results without Φ_1 and (e-h) the results with Φ_1 . Note that the scale of the colour contour is different for each figure. θ and ζ are poloidal and toroidal angle coordinates in the Boozer coordinate system, respectively, and N = 10.

solutions of the ambipolar condition turn out to be qualitatively different between the global and the local simulations and an ion-root is not found by the global simulation.

Now, let us examine the Φ_1 effect on the particle fluxes. In Mollén *et al.* (2018), it is shown that the C⁶⁺ flux in ion-root plasmas is inwardly enhanced by Φ_1 while the H¹⁺ and the He²⁺ fluxes are outwardly driven. A similar result has also been obtained for the C⁶⁺ flux in Velasco *et al.* (2018). Figure 6 shows that our global result for the C⁶⁺ flux is qualitatively analogous to those local results. That is, the C⁶⁺ flux is driven further inwardly, not outwardly. This means that Φ_1 contributes to the impurity accumulation, rather than mitigate it. On the other hand, positive enhancement is seen in the H¹⁺ and He²⁺ fluxes, especially at outer radii r/a > 0.8.

Let us look into the structures of Φ_1 and density variation of each ion species to see which results in the modification of the particle fluxes in figure 6. In figure 7, the phase structures of the normalised Φ_1 and the density variation of each ion species on the flux surface at r/a=0.2 are shown. Figure 7(a-d) represent the results without Φ_1 and figure 7(e-h) the result with Φ_1 . Here ' Φ_1 for the case without Φ_1 ' means Φ_1 constructed from the solutions of the drift-kinetic equation neglecting Φ_1 through (2.34). When the difference in Φ_1 between the cases with and without Φ_1 in this sense becomes significant, the necessity of the self-consistent calculation is



FIGURE 8. From left, structures of Φ_1 , $n_{H,1}$, $n_{He,1}$ and $n_{C,1}$ on the flux surface at r/a=0.5 for the ion-root case. Figures (a-d) represent the results without Φ_1 and (e-h) the results with Φ_1 . Note that the scale of the colour contour is different for each figure. θ and ζ are poloidal and toroidal angle coordinates in the Boozer coordinate system, respectively, and N = 10.



FIGURE 9. From left, structures of Φ_1 , $n_{H,1}$, $n_{He,1}$ and $n_{C,1}$ on the flux surface at r/a=0.8 for the ion-root case. Figures (a-d) represent the results without Φ_1 and (e-h) the results with Φ_1 . Note that the scale of the colour contour is different for each figure. θ and ζ are poloidal and toroidal angle coordinates in the Boozer coordinate system, respectively, and N=10.

indicated. Figures 8 and 9 make the same comparison at r/a = 0.5 and r/a = 0.8, respectively. The amplitude of Φ_1 is smaller than earlier local results overall and the factor $Z_a e \Phi_1/T_a$ is at most the order of 10^{-1} even for C⁶⁺ in this case (for example, $e \Phi_1/T_i = 1.0 \times 10^{-2}$ at r/a = 0.5 roughly amounts to 30 V of Φ_1 in this case). The comparison of the density structures above implies that Φ_1 is too small to strongly modify the spectrum compositions of the ion species over the flux surface.

This can also be seen in figure 10, which represents the Fourier spectra of density variations of the ion species for the cases with and without Φ_1 . The density variations



FIGURE 10. Fourier spectra of $n_{H,1}$ (a,d), $n_{He,1}$ (b,e) and $n_{C,1}$ (c,f) for the ion-root case. Figures (a-c) represent the results without Φ_1 and (d-f) the results with Φ_1 .



FIGURE 11. Fourier spectra of \bar{v}_{E1} (a) and Φ_1 (b) for the ion-root case.

are expanded in real trigonometric series. Only leading modes are plotted and reddish colours are assigned to cosine modes and bluish colours to sine modes. Although the Fourier spectra are evaluated on 50 flux surfaces in FORTEC-3D, only the values at $r/a = 0.0, 0.2, \ldots, 1.0$ are plotted for visualisation purpose. The mode compositions are not largely affected by the inclusion of Φ_1 . One point worth noting is that the phase structure of C⁶⁺ is qualitatively different from those of the other two species. H¹⁺ or He²⁺ mainly consists of stellarator symmetric components, which are preserved under the transformation $(\theta, \zeta) \rightarrow (-\theta, -\zeta)$, as expected for the ions in the $\sqrt{\nu}$ regime (Calvo *et al.* 2018*b*). On the other hand, the main components in the density spectrum of C⁶⁺, which is in the plateau regime, change their signs under the same transformation.

The spectrum of \bar{v}_{E1} is plotted in figure 11 as well as the spectrum of Φ_1 itself. For H^{1+} and He^{2+} , following (2.39), most of the cosine modes in their density spectra couple with the same modes in the \bar{v}_{E1} spectrum having the same signs. These effective couplings result in the outward enhancement of the He^{2+} flux and H^{1+} flux at r/a > 0.8. On the other hand, the sine-(1, 0) mode dominates in the C^{6+} spectrum.



FIGURE 12. (a) The total radial C^{6+} flux, Γ_C , for the case with Φ_1 (red), the v_{E1} -driven part of Γ_C calculated by the integral (green) and the v_{E1} -driven part calculated by the products of the Fourier coefficients (blue). (b) The total radial C^{6+} flux, Γ_C , for the case with Φ_1 (red), the total C^{6+} flux (driven only by \boldsymbol{v}_m) for the case without Φ_1 (green) and the v_m -driven part for the case with Φ_1 (blue).



FIGURE 13. Radial particle fluxes of H¹⁺ (*a*), He²⁺ (*b*) and C⁶⁺ (*c*) for the electron-root case without (red) and with (green) Φ_1 , respectively.

The value of this mode peaks around r/a = 0.6 and the mode couples with \bar{v}_{E1} having the opposite sign and results in the inward enhancement of the particle flux.

In figure 12(*a*), the radial C⁶⁺ flux driven by v_{E1} is plotted as well as the total C⁶⁺ flux for the case with Φ_1 . The red line represents the total C⁶⁺ flux calculated by the integral (1.1). The green line represents the v_{E1} -driven part of the C⁶⁺ flux calculated by the integral (2.35). The blue line is also the v_{E1} -driven part of the C⁶⁺ flux, but it is obtained by the sum of the Fourier coefficient products in the form of (2.39). The impact of Φ_1 on the v_m -driven flux is also shown in figure 12(*b*). The green line represents the C⁶⁺ flux driven by the magnetic drift for the case without Φ_1 , i.e. the total C⁶⁺ for the case without Φ_1 . The blue line represents the C⁶⁺ flux driven by v_m for the case with Φ_1 . It can been seen that the v_m -driven flux is also enhanced by the inclusion of Φ_1 . This is because, as shown in figure 10, Φ_1 amplifies the sine-(1, 0) mode in the C⁶⁺ spectrum and hence strengthens the opposite-sign couplings between the magnetic drift and the C⁶⁺ distribution function as well. Here the sine-(1, 0) component of v_m represents the grad-*B* drift by the toroidicity of the magnetic field. Therefore, at least in this case, the presence of Φ_1 is unfavourable for impurity accumulation avoidance.

4.2. Electron-root case

In figure 13, the radial particle fluxes with and without Φ_1 for the electron-root case are compared. Regardless of the species, the radial particle flux is outwardly



FIGURE 14. From left, structures of Φ_1 , $n_{H,1}$, $n_{He,1}$ and $n_{C,1}$ on the flux surface at r/a = 0.2 for the electron-root case. Figures (a-d) represent the results without Φ_1 and (e-h) the results with Φ_1 . Note that the scale of the colour contour is different for each figure. θ and ζ are poloidal and toroidal angle coordinates in the Boozer coordinate system, respectively, and N = 10.



FIGURE 15. From left, structures of Φ_1 , $n_{H,1}$, $n_{He,1}$ and $n_{C,1}$ on the flux surface at r/a = 0.8 for the electron-root case. Figures (a-d) represent the results without Φ_1 and (e-h) the results with Φ_1 . Note that the scale of the colour contour is different for each figure. θ and ζ are poloidal and toroidal angle coordinates in the Boozer coordinate system, respectively, and N = 10.

enhanced by Φ_1 . The enhancement is largest for C⁶⁺ and the existence of an outward convection is indicated. However, the enhancements of the H¹⁺ and He²⁺ fluxes are not negligible as well. This result implies that the impact of Φ_1 is too large to be ignored in the calculation of the ambipolar condition. Thus, it is expected that an electron-root slightly different from that used in this work would be found by calculation including Φ_1 .

The reason for the strong outward enhancement can be seen in the phase structures of density variations. The spatial structures of Φ_1 and n_{I1} at r/a = 0.2 and r/a = 0.8 for this case are illustrated in figures 14 and 15, respectively. The amplitude of n_{I1}



FIGURE 16. Fourier spectra of $n_{H,1}$ (*a,d*), $n_{He,1}$ (*b,e*) and $n_{C,1}$ (*c,f*) for the electronroot case. Figures (*a*-*c*) represent the results without Φ_1 and (*d*-*f*) the results with Φ_1 .



FIGURE 17. Fourier spectra of \bar{v}_{E1} (a) and Φ_1 (b) for the electron-root case with Φ_1 .

and hence of Φ_1 are at the same level as those in the ion-root case. Also, the phase structures of Φ_1 , H^{1+} and He^{2+} seemingly look similar between the ion-root and the electron-root cases. However, the C⁶⁺ density structure is distinctively different between the two cases. By comparing figures 9(d,h) and 15(d,h), it can be seen that the C⁶⁺ density phase is inverted in the θ -direction.

This can also be seen in terms of Fourier spectra of the density variations and \bar{v}_E shown in figures 16 and 17, respectively. As shown in figure 16, the signs of the sine-(1, 0) modes in the density spectra are inverted from those in the ion-root case. This is true not only for C⁶⁺, but also for H¹⁺ and He²⁺. Further, this mode is amplified by the inclusion of Φ_1 . Figure 17 shows that the sine-(1, 0) mode in the \hat{v}_E spectrum, which comes from the cosine-(1, 0) mode in the Φ_1 spectrum, is negative and is a major component for this case as well. Thus, the inversion of the sine-(1, 0) modes in the density spectra leads to strong same-sign couplings and contributes to the outward radial particle fluxes. This sign inversion is not a consequence of the inclusion of Φ_1 but, rather, it seems to be a consequence of the presence of the positive radial electric field.



FIGURE 18. (a) The total radial C⁶⁺ flux, Γ_C , for the case with Φ_1 (red), the v_{E1} -driven part of Γ_C calculated by the integral (green) and the v_{E1} -driven part calculated by the products of the Fourier coefficients (blue). (b) The total radial C⁶⁺ flux, Γ_C , for the case with Φ_1 (red), the total C⁶⁺ flux (driven only by v_m) for the case without Φ_1 (green) and the v_m -driven part for the case with Φ_1 (blue).

In figure 18, the contributions of the v_{E1} -driven part (a) and the v_m -driven part (b) to the total radial C⁶⁺ flux are separately represented. The red lines in the figures represent the total C⁶⁺ flux for the case with Φ_1 . The green and blue lines in figure 18(a) represent the v_{E1} -driven part of the C⁶⁺ flux, each calculated by the integral and the sum of Fourier coefficients, respectively. In figure 18(b), the green line represents the total C⁶⁺ flux for the case with Φ_1 , which is driven only by v_m . The blue line is the v_m -driven flux for the case with Φ_1 . As for the ion-root case, the v_m -driven flux is reinforced by Φ_1 as well. The primary cause of this enhancement is the amplification of the sine-(1, 0) mode in the C⁶⁺ spectrum by Φ_1 (see figure 16c, f). As the result, the v_m -driven part becomes at most an order of magnitude larger than the v_{E1} -driven part.

4.3. Discussion of the result

For the ion-root case, a result analogous to previous local results has been obtained: the C⁶⁺ flux is driven further inwardly, not outwardly by Φ_1 . This enhancement is mainly caused by the couplings of the sine-(1, 0) modes between the C⁶⁺ density spectrum and the radial drift velocities. Φ_1 amplifies this mode and the opposite-sign couplings of the sine-(1, 0) modes result in the further inward flux. For the electronroot case, on the other hand, the radial particle flux is outwardly enhanced regardless of the species mainly due to the same-sign couplings of the sine-(1, 0) modes.

The phase structure of Φ_1 is mainly determined by the spatial distribution of main ions. The main components in the Φ_1 spectrum are then determined by the magnetic field configuration. The spectra of H¹⁺ or light impurities such as He²⁺ in low-collisionality regimes mainly consist of stellarator symmetric components. Thus, stellarator anti-symmetric modes become dominant in the spectrum of \bar{v}_{E1} . On the other hand, stellarator anti-symmetric components were found to be dominant in the C⁶⁺ density spectrum, and these components tend to couple well with both the magnetic drift and the $E \times B$ drift. The important question here is 'with which sign do the leading modes, in particular the sine (1, 0)-modes, of the impurity ions couple with the radial drift velocities?'. An induction from our results implies that the sign of E_r is a key factor to determine the signs of the sine-(1, 0) modes and, for ion-root plasmas, opposite-sign couplings are realised. To avoid those effective

negative couplings or turn them into positive ones, either the phase structure of Φ_1 or the impurity density has to be strongly modified. However, we found that Φ_1 only increases the absolute value of the sine-(1, 0) mode in the C⁶⁺ density spectrum and does not seem to affect its sign. Increasing the amplitude of Φ_1 is unlikely to make a qualitative difference either. Thus, our conclusion is that it is difficult for Φ_1 to mitigate the inward neoclassical C⁶⁺ flux, let alone drive it in the outward direction in ion-root plasmas unless a strong particle or torque source exists.

The argument above ignores the contribution of kinetic electrons and it is possible that their contribution modifies the Φ_1 structure (García-Regaña *et al.* 2018). Although it is worth examining, however, it is doubtful that the impact of kinetic electrons can be strong enough to reverse the conclusion. On the other hand, our result does not simply support the electron-root scenario either. In the present study, we have demonstrated that Φ_1 is indeed a necessary factor to study the role of neoclassical transport in the formation of impurity hole, even in a global simulation model. Although it is an important finding that the global simulation alters the ambipolar- E_r profile from the prediction by local codes, more detailed and accurate calculation, in which the ambipolar- E_r profile is determined without neglecting the effect of Φ_1 on the radial transport, remains for future research.

5. Conclusion

We have extended and enabled a global neoclassical code FORTEC-3D to simulate impurity transport including the effect of Φ_1 . One of the motivations for this extension is to investigate the impurity hole phenomenon. In this study, we found that the impact of Φ_1 on the radial impurity transport in unoptimised devices such as LHD is mainly determined by the couplings of the sine-(1, 0) modes between the radial components of the drift velocities and the impurity distribution. Further, the direction of the enhancement of radial particle fluxes by Φ_1 depends on the ambipolar- E_r profile. As the result, while our global result provides another support that Φ_1 has non-negligible impact on impurity transport, it was shown that Φ_1 does not drive impurities outwardly in the ion-root plasma. Instead, a possibility that the ambipolar radial electric field in the impurity hole plasma is positive has emerged.

In this study, it has been assumed that the global effects are crucial, since the plasma profile for our ion-root case belongs in the parameter region where the global effects become most remarkable. However, we cannot distinguish between the contributions of the radial component and the tangential component of v_m only in the present global result. Therefore, although we can at least conclude that considering the effects of v_m in the guiding-centre orbit is essential for studying transport in impurity hole plasmas, we cannot tell to what extent the global effects are necessary. Comparison of calculation results between the global model and the local model with the tangential component of the magnetic drift will be made in a separate study.

We also found that there are some points to be improved in our calculation method. In particular, simultaneous calculation of global neoclassical transport and ambipolar radial electric field in multi-ion-species plasmas including Φ_1 is needed. The impact of kinetic electrons may be non-negligible as well. Performing the computation including the effects mentioned above with reasonable numerical resource remains as a future work. On the other hand, the existing experimental data on the ambipolar radial electric fields in impurity hole plasmas are not sufficient as well. Thus, more efforts to accumulate reliable data in both numerical simulation and experimental measurements are needed to reveal the mechanism behind the impurity hole phenomenon.

Acknowledgements

This work is performed under the auspices of the NIFS Collaborative Research Program, nos. NIFS18KNST132 and NIFS16KNXN315. The simulations in this article were carried out on the Plasma Simulator in NIFS, Japan and on the JFRS-1 supercomputer system at the Computational Simulation Centre of the International Fusion Energy Research Centre (IFERC-CSC) in Rokkasho Fusion Institute of QST (Aomori, Japan).

Editor Peter Catto thanks the referees for their advice in evaluating this article.

Appendix A. Equations solved by FORTEC-3D

A.1. Derivation of the 5D drift-kinetic equation

The model used in FORTEC-3D is based on Littlejohn's guiding-centre Lagrangian (Littlejohn 1983)

$$L = \frac{1}{\epsilon} (Z_a e A + \epsilon m_a v_{\parallel} b) \cdot \dot{X} + \epsilon \frac{m_a}{Z_a e} \mu \dot{\xi} - H, \qquad (A1)$$

where the small parameter ϵ is temporarily introduced and the guiding-centre coordinates X, the parallel velocity v_{\parallel} , the magnetic moment μ and the gyrophase ξ are chosen as the independent variables. The Hamiltonian H is given by

$$H = \frac{1}{2}m_a v_{\parallel}^2 + \mu B + Z_a e\Phi \tag{A2}$$

and the Jacobian for the guiding-centre velocity variables is

$$D \equiv \frac{B_{\parallel}^*}{m_a} = \frac{1}{m_a} \left(B + \epsilon \frac{m_a v_{\parallel}}{Z_a e} \boldsymbol{b} \cdot \boldsymbol{\nabla} \times \boldsymbol{b} \right).$$
(A 3)

For a case with $\partial A/\partial t = \partial A^*/\partial t = 0$, the Lagrangian (A1) gives the equations of motion

$$\frac{\mathrm{d}X}{\mathrm{d}t} = \frac{\boldsymbol{B}^*}{B^*_{\parallel}} \boldsymbol{v}_{\parallel} + \frac{\epsilon}{Z_a e B^*_{\parallel}} \boldsymbol{b} \times (\mu \nabla B + Z_a e \nabla \Phi), \tag{A4}$$

$$\frac{\mathrm{d}v_{\parallel}}{\mathrm{d}t} = -\frac{\dot{X}}{m_a v_{\parallel}} \cdot (\mu \nabla B + Z_a e \nabla \Phi), \tag{A5}$$

$$\frac{\mathrm{d}\mu}{\mathrm{d}t} = 0 \tag{A6}$$

and (A4) is equivalent to another expression

$$v_{\parallel}\boldsymbol{b} + \frac{\epsilon}{Z_a e B_{\parallel}^*} \boldsymbol{b} \times (m_a v_{\parallel}^2 \boldsymbol{b} \cdot \nabla \boldsymbol{b} + \mu \nabla B + Z_a e \nabla \Phi), \tag{A7}$$

to the order of $O(\epsilon^2)$, which makes it easier to see the correspondence of each term between different trajectory models (see e.g. Landreman *et al.* 2014; Huang *et al.* 2017). The parameter ϵ is omitted in the rest of this paper. The drift-kinetic equation with the phase-space variables $\mathbf{Z} = (X, v_{\parallel}, \mu)$ can be written as

$$\frac{\partial f_{a1}}{\partial t} + \dot{X} \cdot \nabla f_{a1} + \dot{v}_{\parallel} \frac{\partial f_{a1}}{\partial v_{\parallel}} - C(f_{a1}) = -\dot{X} \cdot \nabla f_{a0} - \dot{v}_{\parallel} \frac{\partial f_{a0}}{\partial v_{\parallel}}, \tag{A8}$$

where $C(f_{a1})$ is the linearised collision operator. Substituting (A 5) into the last term on the right-hand side,

$$-\dot{v}_{\parallel}\frac{\partial f_{a0}}{\partial v_{\parallel}} = \dot{v}_{\parallel}\frac{m_{a}v_{\parallel}}{T_{a}}f_{a0},\tag{A9}$$

which yields

$$-\dot{v}_{\parallel}\frac{\partial f_{a0}}{\partial v_{\parallel}} = -\frac{1}{T_a}\dot{X} \cdot (Z_a e \nabla \Phi_0 + Z_a e \nabla \Phi_1 + \mu \nabla B) f_{a0}, \qquad (A\,10)$$

and the last two terms in the bracket cancel with the last term in

$$-\dot{X} \cdot \nabla f_{a0} = -\dot{X} \cdot \nabla r \left[\frac{n_{a0}'}{n_{a0}} + \left(\frac{m_a v_{\parallel}^2}{2T_a} + \frac{\mu B}{T_a} - \frac{3}{2} + \frac{Z_a e \Phi_1}{T_a} \right) \frac{T_a'}{T_a} \right] f_{a0} + \frac{1}{T_a} \dot{X} \cdot (Z_a e \nabla \Phi_1 + \mu \nabla B) f_{a0},$$
(A11)

leaving the right-hand side of (2.6) other than the collision term.

A.2. Expressions in the Boozer coordinates

Using the relation

$$\boldsymbol{b} \times \boldsymbol{V} = \frac{1}{B} (I \nabla \theta + G \nabla \zeta) \times (V_r \nabla r + V_\theta \nabla \theta + V_\zeta \nabla \zeta)$$
$$= \frac{1}{B \sqrt{g}} [(I V_\zeta - G V_\theta) \boldsymbol{e}_r + V_r (G \boldsymbol{e}_\theta - I \boldsymbol{e}_\zeta)], \qquad (A12)$$

for an arbitrary vector in the configuration space V and

$$\nabla \times \boldsymbol{b} = \nabla \times \frac{\boldsymbol{B}}{B}$$
$$= \frac{1}{B} \nabla \times \boldsymbol{B} + \frac{1}{B^2} \boldsymbol{B} \times \nabla \boldsymbol{B}, \qquad (A\,13)$$

we can express B^* as

$$B^{*} = B + \frac{m_{a}v_{\parallel}}{Z_{a}e} \nabla \times b$$

= $\frac{1}{\sqrt{g}} \frac{\mathrm{d}\psi}{\mathrm{d}r} (\iota e_{\theta} + e_{\zeta})$
+ $\frac{m_{a}v_{\parallel}}{Z_{a}eB\sqrt{g}} \left[\frac{1}{B} \left(I \frac{\partial B}{\partial \zeta} - G \frac{\partial B}{\partial \theta} \right) e_{r} + \left(\frac{B'}{B}G - G' \right) e_{\theta} - \left(\frac{B'}{B}I - I' \right) e_{\zeta} \right]$ (A 14)

and B^*_{\parallel} as

$$B_{\parallel}^{*} = B + \frac{m_{a}v_{\parallel}}{Z_{a}e} \frac{GI' - IG'}{(\psi'G + \chi'I)}.$$
 (A 15)

Then, taking the dot products of (A4) with ∇r , $\nabla \theta$ and $\nabla \zeta$ yields

$$\begin{aligned} \frac{\mathrm{d}r}{\mathrm{d}t} &= \frac{1}{Z_a e B B_{\parallel}^* \sqrt{g}} \left[\left(\mu + \frac{m_a v_{\parallel}^2}{B} \right) \left(I \frac{\partial B}{\partial \zeta} - G \frac{\partial B}{\partial \theta} \right) + Z_a e \left(I \frac{\partial \Phi}{\partial \zeta} - G \frac{\partial \Phi}{\partial \theta} \right) \right] \\ &= \frac{\delta}{\gamma} \left(I \frac{\partial B}{\partial \zeta} - G \frac{\partial B}{\partial \theta} \right) + \frac{Z_a e}{\gamma} \left(I \frac{\partial \Phi}{\partial \zeta} - G \frac{\partial \Phi}{\partial \theta} \right), \end{aligned} \tag{A 16} \\ \frac{\mathrm{d}\theta}{\mathrm{d}t} &= \frac{v_{\parallel}}{\sqrt{g} B_{\parallel}^*} \left[\psi' \iota + \frac{m_a v_{\parallel}}{Z_a e B} \left(\frac{B'}{B} G - G' \right) \right] + \frac{G}{Z_a e B B_{\parallel}^* \sqrt{g}} (\mu B' + Z_a e \Phi') \\ &= \frac{G}{\gamma} (\delta B' + Z_a e \Phi') + \frac{v_{\parallel} B}{\gamma} \left(Z_a e \chi' - \frac{m_a v_{\parallel}}{B} G' \right), \end{aligned} \tag{A 17} \\ \frac{\mathrm{d}\zeta}{\mathrm{d}t} &= \frac{v_{\parallel}}{\sqrt{g} B_{\parallel}^*} \left[\psi' - \frac{m_a v_{\parallel}}{Z_a e B} \left(\frac{B'}{B} I - I' \right) \right] - \frac{I}{Z_a e B B_{\parallel}^* \sqrt{g}} (\mu B' + Z_a e \Phi') \\ &= -\frac{I}{\gamma} (\delta B' + Z_a e \Phi') + \frac{v_{\parallel} B}{\gamma} \left(Z_a e \psi' + \frac{m_a v_{\parallel}}{B} I' \right), \end{aligned} \tag{A 18}$$

respectively, and, using these expressions, we obtain the expression of the derivative of v_{\parallel} as

$$\frac{\mathrm{d}v_{\parallel}}{\mathrm{d}t} = -\frac{v_{\parallel}}{B\gamma} \left[Z_{a}e \left(I \frac{\partial B}{\partial \zeta} - G \frac{\partial B}{\partial \theta} \right) \Phi' - \mu \left(I \frac{\partial \Phi}{\partial \zeta} - G \frac{\partial \Phi}{\partial \theta} \right) B' \right]
+ \frac{Z_{a}eB}{m_{a}\gamma} \left[\left(\frac{m_{a}v_{\parallel}}{Z_{a}eB}G' - \chi' \right) \left(\mu \frac{\partial B}{\partial \theta} + Z_{a}e \frac{\partial \Phi}{\partial \theta} \right)
- \left(\frac{m_{a}v_{\parallel}}{Z_{a}eB}I' + \psi' \right) \left(\mu \frac{\partial B}{\partial \zeta} + Z_{a}e \frac{\partial \Phi}{\partial \zeta} \right) \right],$$
(A 19)

where δ and γ are defined by (2.21) and (2.22), respectively.

A.3. Verification of (2.13)

In appendix B of García-Regaña et al. (2017), it is demonstrated that the relation

$$\left\langle \int \mathrm{d}^3 v \, \boldsymbol{v}_d \cdot \boldsymbol{\nabla} r f_{a0} \right\rangle = 0 \tag{A 20}$$

still holds when f_{a0} is chosen as $f_{a0} = f_{aM} \exp(-Z_a e \Phi_1/T_a)$ instead of $f_{a0} = f_{aM}$. Although the definitions of the drift velocities in our global model differ from those in the models used in García-Regaña *et al.* (2017) and other local studies, particularly, $B_{\parallel}^*(X, v)$ appears in the denominators of the drift velocities, it can be shown that the relation (2.13) is unaffected.

The demonstration of (A 20) for our model is analogous to that in García-Regaña *et al.* (2017). In our coordinate system, (A 20) or (2.13) can be expressed as

$$\Gamma_{a0} = \frac{2\pi}{V'} \oint \sqrt{g} \,\mathrm{d}\theta \,\mathrm{d}\zeta \,\int D \,\mathrm{d}v_{\parallel} \,\mathrm{d}\mu \frac{\mathrm{d}r}{\mathrm{d}t} f_{a0}. \tag{A21}$$



FIGURE 19. Contributions of f_1 (red) and f_0 (green) to the radial carbon flux.

Thus, substituting (A 16) into (A 21) cancels the Jacobian, $D\sqrt{g}$, and leaves

$$\Gamma_{a0} = \frac{2\pi}{Z_a e m_a} \frac{1}{V'} \oint d\theta d\zeta \int dv_{\parallel} d\mu \\ \times \frac{1}{B} \left[\left(\mu + \frac{m_a v_{\parallel}^2}{B} \right) \left(I \frac{\partial B}{\partial \zeta} - G \frac{\partial B}{\partial \theta} \right) + Z_a e \left(I \frac{\partial \Phi}{\partial \zeta} - G \frac{\partial \Phi}{\partial \theta} \right) \right] f_{a0}.$$
(A 22)

We can then perform the integrals in the velocity space to have

$$\Gamma_{a0} = \frac{n_{a0}}{Z_{a}e} \frac{1}{V'} \oint d\theta d\zeta \frac{1}{B^{2}} \\ \times \left[\frac{2T_{a}}{B} \left(I \frac{\partial B}{\partial \zeta} - G \frac{\partial B}{\partial \theta} \right) + Z_{a}e \left(I \frac{\partial \Phi}{\partial \zeta} - G \frac{\partial \Phi}{\partial \theta} \right) \right] e^{-Z_{a}e\Phi_{1}/T_{a}}$$
(A 23)

and this expression can be transformed into

$$\Gamma_{a0} = -\frac{n_{a0}}{Z_a e} \frac{T_a}{V'} \oint d\theta d\zeta \left(I \frac{\partial}{\partial \zeta} - G \frac{\partial}{\partial \theta} \right) \left(\frac{1}{B^2} e^{-Z_a e \Phi_1 / T_a} \right).$$
(A24)

Now, it can be seen that the integrals in θ and ζ vanish due to the periodic dependence of *B* and Φ_1 on those angle variables. In figure 19, Γ_{a0} for C⁶⁺ is plotted with the f_1 -part defined by (1.1). The numerical error is within the tolerance compared with the f_1 -part.

Appendix B. Method for obtaining continuous distribution functions

In this study, continuous forms of physical quantities such as the distribution function, f_{a1} , the density variation, n_{a1} , and the non-uniform part of electrostatic potential, Φ_1 , are obtained by the following method. First, the Boozer configuration space is partitioned into small cells, where the *r*-direction is divided into 50 and both the θ - and ζ -directions are divided into 40, respectively. Then, at each calculation step, the quantity, of which the continuous distribution we want, is averaged in each cell to obtain its coarse-grained distribution. After several time steps, the coarse-grained distribution is averaged over that period. Finally, the continuous spectrum on each



FIGURE 20. The ratio p_{I1}/p_{I0} at r/a = 0.2 (a), r/a = 0.5 (b) and r/a = 0.8 (c) for the ion-root case with Φ_1 , respectively.

flux surface is obtained by Fourier transformation along angle coordinates, θ and ζ . In the *r*-direction, each Fourier component is interpolated with a third-order spline function. Using this spectrum, smooth functions can be constructed.

Regarding this method, another approximation is used for some quantities. In global calculation, the constraint

$$\langle f_{a1} \rangle = 0 \tag{B1}$$

cannot usually be imposed due to the radial drift. Thus, the flux surface average of the density variation is not usually equal to zero, that is, its cosine-(0, 0) mode, $\tilde{n}_{a1}^{00(c)}$, does not vanish:

$$\langle n_{a1} \rangle = \tilde{n}_{a1}^{00(c)} \neq 0. \tag{B2}$$

However, FORTEC-3D adopts adaptive source and sink terms in the drift-kinetic equation (Huang *et al.* 2017), so that $\tilde{n}_{a1}^{00(c)} \ll n_{a0}$ is kept. We thus can ignore $\tilde{n}_{a1}^{00(c)}$ and assume that the cosine-(0, 0) mode of Φ_1 is zero as well:

$$\langle \Phi_1 \rangle = \tilde{\Phi}_1^{00(c)} = 0. \tag{B3}$$

Appendix C. Pressure anisotropy

One may be concerned that the presence of appreciable density variations may be inconsistent with the assumption that the pressure is a flux-surface function in solving the magnetohydrodynamic (MHD) equilibrium. To address this concern, the ratio between the uniform part and the non-uniform part of the total ion pressure

$$\frac{p_{I1}}{p_{I0}} = \frac{\sum_{I} n_{I1}}{\sum_{I} n_{I0}}$$
(C1)

is illustrated in figure 20 (the temperatures are assumed to be the same). It can be seen that the ratio is of the order of 10^{-2} or less than that. Thus, while the pressure anisotropy is large enough to generate numerically appreciable Φ_1 , it is small enough not to affect the MHD equilibrium condition in the macroscopic scale.

REFERENCES

BOOZER, A. H. 1981 Plasma equilibrium with rational magnetic surfaces. *Phys. Fluids* 24 (11), 1999–2003.

BRUNNER, S., VALEO, E. & KROMMES, J. A. 1999 Collisional delta-f scheme with evolving background for transport time scale simulations. *Phys. Plasmas* 6 (12), 4504–4521.

- BULLER, S., SMITH, H., HELANDER, P., MOLLÉN, A., NEWTON, S. & PUSZTAI, I. 2018 Collisional transport of impurities with flux-surface varying density in stellarators. J. Plasma Phys. 84 (4), 905840409.
- BURHENN, R., FENG, Y., IDA, K., MAASSBERG, H., MCCARTHY, K. J., KALININA, D., KOBAYASHI, M., MORITA, S., NAKAMURA, Y., NOZATO, H. et al. 2009 On impurity handling in high performance stellarator/heliotron plasmas. Nucl. Fusion 49 (6), 065005.
- CALVO, I., PARRA, F. I., VELASCO, J. L. & ALONSO, J. A. 2017 The effect of tangential drifts on neoclassical transport in stellarators close to omnigeneity. *Plasma Phys. Control. Fusion* 59 (5), 055014.
- CALVO, I., PARRA, F. I., VELASCO, J. L., ALONSO, J. A. & GARCÍA-REGAÑA, J. M. 2018a Stellarator impurity flux driven by electric fields tangent to magnetic surfaces. *Nucl. Fusion* 58 (12), 124005.
- CALVO, I., PARRA, F. I., VELASCO, J. L. & GARCÍA-REGAÑA, J. M. 2020 Impact of main ion pressure anisotropy on stellarator impurity transport. *Nucl. Fusion* **60** (1), 016035.
- CALVO, I., VELASCO, J., PARRA, F., ALONSO, J. & GARCÍA-REGAÑA, J. 2018b Electrostatic potential variations on stellarator magnetic surfaces in low collisionality regimes. J. Plasma Phys. 84 (4), 905840407.
- ESTRADA, T., SÁNCHEZ, E., GARCÍA-REGAÑA, J. M., ALONSO, J. A., ASCASÍBAR, E., CALVO, I., CAPPA, A., CARRALERO, D., HIDALGO, C., LINIERS, M. *et al.* 2019 Turbulence and perpendicular plasma flow asymmetries measured at TJ-II plasmas. *Nucl. Fusion* **59** (7), 076021.
- FUJITA, K., SATAKE, S., KANNO, R., NUNAMI, M., NAKATA, M. & GARCÍA-REGAÑA, J. M. 2019 Global effects on the variation of ion density and electrostatic potential on the flux surface in helical plasmas. *Plasma Fusion Res.* 14, 3403102.
- GARCÍA-REGAÑA, J. M., BEIDLER, C. D., KLEIBER, R., HELANDER, P., MOLLÉN, A., ALONSO, J. A., LANDREMAN, M., MAASSBERG, H., SMITH, H. M., TURKIN, Y. et al. 2017 Electrostatic potential variation on the flux surface and its impact on impurity transport. Nucl. Fusion 57 (5), 056004.
- GARCÍA-REGAÑA, J. M., ESTRADA, T., CALVO, I., VELASCO, J. L., ALONSO, J. A., CARRALERO, D., KLEIBER, R., LANDREMAN, M., MOLLÉN, A., SÁNCHEZ, E. et al. 2018 On-surface potential and radial electric field variations in electron root stellarator plasmas. *Plasma Phys. Control. Fusion* 60 (10), 104002.
- GARCÍA-REGAÑA, J. M., KLEIBER, R., BEIDLER, C. D., TURKIN, Y., MAASSBERG, H. & HELANDER, P. 2013 On neoclassical impurity transport in stellarator geometry. *Plasma Phys. Control. Fusion* 55 (7), 074008.
- HELANDER, P., NEWTON, S. L., MOLLÉN, A. & SMITH, H. M. 2017 Impurity transport in a mixed-collisionality stellarator plasma. *Phys. Rev. Lett.* 118 (15), 155002.
- HUANG, B., SATAKE, S., KANNO, R., SUGAMA, H. & MATSUOKA, S. 2017 Benchmark of the local drift-kinetic models for neoclassical transport simulation in helical plasmas. *Phys. Plasmas* 24 (2), 022503.
- IDA, K., YOSHINUMA, M., OSAKABE, M., NAGAOKA, K., YOKOYAMA, M., FUNABA, H., SUZUKI, C., IDO, T., SHIMIZU, A., MURAKAMI, I. *et al.* 2009 Observation of an impurity hole in a plasma with an ion internal transport barrier in the large helical device. *Phys. Plasmas* 16 (5), 056111.
- IDO, T., SHIMIZU, A., NISHIURA, M., NAGAOKA, K., YOKOYAMA, M., IDA, K., YOSHINUMA, M., TOI, K., ITOH, K., NAKANO, H. *et al.* 2010 Experimental study of radial electric field and electrostatic potential fluctuation in the large helical device. *Plasma Phys. Control. Fusion* 52 (12), 124025.
- LANDREMAN, M., SMITH, H. M., MOLLÉN, A. & HELANDER, P. 2014 Comparison of particle trajectories and collision operators for collisional transport in nonaxisymmetric plasmas. *Phys. Plasmas* 21 (4), 042503.
- LITTLEJOHN, R. G. 1983 Variational principles of guiding centre motion. J. Plasma Phys. 29 (1), 111–125.
- MATSUOKA, S., SATAKE, S., KANNO, R. & SUGAMA, H. 2015 Effects of magnetic drift tangential to magnetic surfaces on neoclassical transport in non-axisymmetric plasmas. *Phys. Plasmas* 22 (7), 072511.

- MOLLÉN, A., LANDREMAN, M., SMITH, H. M., GARCÍA-REGAÑA, J. M. & NUNAMI, M. 2018 Flux-surface variations of the electrostatic potential in stellarators: impact on the radial electric field and neoclassical impurity transport. *Plasma Phys. Control. Fusion* 60 (8), 084001.
- NAKAMURA, Y., TAMURA, N., YOSHINUMA, M., SUZUKI, C., YOSHIMURA, S., KOBAYASHI, M., YOKOYAMA, M., NUNAMI, M., NAKATA, M., NAGAOKA, K. *et al.* 2017 Strong suppression of impurity accumulation in steady-state hydrogen discharges with high power NBI heating on LHD. *Nucl. Fusion* **57** (5), 056003.
- NEWTON, S., HELANDER, P., MOLLÉN, A. & SMITH, H. 2017 Impurity transport and bulk ion flow in a mixed collisionality stellarator plasma. J. Plasma Phys. 83 (5), 905830505.
- NUNAMI, M., NAKATA, M., SATO, M., TODA, S., YAMAGUCHI, H., SUGAMA, H. & YOKOYAMA, M. 2017 Kinetic simulation studies on multi-ion-species plasma transport in helical systems. In 27th IAEA Fusion Energy Conference (FEC 2018) 22–27 October 2018, Gandhinagar (nearest Airport: Ahmedabad), India. https://www.iaea.org/events/fec-2018.
- NUNAMI, M., NAKATA, M., TODA, S. & SUGAMA, H. 2020 Gyrokinetic simulations for turbulent transport of multi-ion-species plasmas in helical systems. *Phys. Plasmas* 27 (5), 052501.
- NUNAMI, M., SATAKE, S., FUJITA, K., YAMAGUCHI, H., MATSUOKA, S., SATO, M., TODA, S. & SUGAMA, H. 2019 Particle balance in turbulent and neoclassical transport of helical plasmas. In 22nd ISHW (22nd International Stellarator and Heliotron Workshop) September 23–27, 2019. University of Wisconsin-Madison.
- NUNAMI, M., SUGAMA, H., VELASCO, J. L., YOKOYAMA, M., SATO, M., NAKATA, M. & SATAKE, S. 2016 Anomalous and neoclassical transport of hydrogen isotope and impurity ions in LHD plasmas. In 26th IAEA Fusion Energy Conference, 17–22 October 2016, Kyoto, Japan. Conference ID: 48315 (CN-234). https://www-pub.iaea.org/iaeameetings/48315/26th-IAEA-Fusion-Energy-Conference.
- PEDROSA, M. A., ALONSO, J. A., GARCÍA-REGAÑA, J. M., HIDALGO, C., VELASCO, J. L., CALVO, I., KLEIBER, R., SILVA, C. & HELANDER, P. 2015 Electrostatic potential variations along flux surfaces in stellarators. *Nucl. Fusion* 55 (5), 052001.
- SATAKE, S., IDOMURA, Y., SUGAMA, H. & WATANABE, T.-H. 2010 Benchmark test of drift-kinetic and gyrokinetic codes through neoclassical transport simulations. *Comput. Phys. Comm.* 181 (6), 1069–1076.
- SATAKE, S., KANNO, R. & SUGAMA, H. 2008 Development of a non-local neoclassical transport code for helical configurations. *Plasma Fusion Res.* **3**, S1062.
- SATAKE, S., NAKATA, M., PIANPANIT, T., SUGAMA, H., NUNAMI, M., MATSUOKA, S., ISHIGURO, S. & KANNO, R. 2020 Benchmark of a new multi-ion-species collision operator for δf Monte Carlo neoclassical simulation. *Comput. Phys. Commun.* **255**, 107249.
- SUGAMA, H., MATSUOKA, S., SATAKE, S., NUNAMI, M. & WATANABE, T.-H. 2019 Improved linearized model collision operator for the highly collisional regime. *Phys. Plasmas* 26 (10), 102108.
- SUGAMA, H., WATANABE, T.-H. & NUNAMI, M. 2009 Linearized model collision operators for multiple ion species plasmas and gyrokinetic entropy balance equations. *Phys. Plasmas* 16 (11), 112503.
- VELASCO, J.L., CALVO, I., GARCÍA-REGAÑA, J.M., PARRA, F.I., SATAKE, S. & ALONSO, J.A. 2018 Large tangential electric fields in plasmas close to temperature screening. *Plasma Phys. Control. Fusion* **60** (7), 074004.
- VELASCO, J. L., CALVO, I., PARRA, F. I., ALONSO, J. A., CARRALERO, D., ESTRADA, T., GARCÍA-REGAÑ, J. M., HUANG, X., MORITA, S., SATAKE, S. et al. 2019 Fast calculation of neoclassical transport of energy and impurities in arbitrary stellarator geometry with the code KNOSOS. In 22nd ISHW (22nd International Stellarator and Heliotron Workshop) September 23–27, 2019. University of Wisconsin-Madison.
- VELASCO, J. L., CALVO, I., PARRA, F. I. & GARCÍA-REGANA, J. M. 2020 KNOSOS: a fast orbit-averaging neoclassical code for arbitrary stellarator geometry. J. Comput. Phys. 109512. doi:10.1016/j.jcp.2020.109512.
- VELASCO, J. L., CALVO, I., SATAKE, S., ALONSO, A., NUNAMI, M., YOKOYAMA, M., SATO, M., ESTRADA, T., FONTDECABA, J. M., LINIERS, M. et al. 2017 Moderation of neoclassical impurity accumulation in high temperature plasmas of helical devices. Nucl. Fusion 57, 016016.

- WANG, W. X., NAKAJIMA, N., OKAMOTO, M. & MURAKAMI, S. 1999 A new f method for neoclassical transport studies. *Plasma Phys. Control. Fusion* **41** (9), 1091.
- WHITE, R. B. 2014 The Theory of Toroidally Confined Plasmas, 3rd edn. World Scientific.
- YOSHINUMA, M., IDA, K. & YOKOYAMA, M. 2010 Impurity transport of ion ITB plasmas on LHD. *Tech. Rep.*, National Institute for Fusion Science.